

Physics of Nuclear Fusion

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- Fusion in the stars
- Fusion research on earth
 - Inertial Confinement Fusion (ICF)
 - Magnetic Confinement Fusion (MCF)
- Status of fusion research and outlook

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The energy source of the stars



- The power flux arriving on earth is 1.4 kW/m² (above the atmosphere, without absoprtion).
- The sun produces continously energy, with a total power of 3.6•10¹⁷ GW.
- In doing so, it converts per second 600 Mio. tons of hydrogen into 596 Mio. tons of helium.



NASA, Skylab space station December 19, 1973, solar flare reaching 588 000 km off solar surface

Nuclear energy: Fission/Fusion



Nuclear reactions: potential energy





- Nuclear forces (strong interaction) act only over distances in the order of the nucleus dimensions (fm).
- Otherwise, the repulsive Coulomb force dominates
 ⇒ Potential wall: some 100 keV, impossible to overcome!
- 1928, Gamov explained α-decay with tunneling-effect (Q.M.): probability function is a spatially decaying wave function with finite values for r < r_n,
 - \Rightarrow finite probability for tunneling through the Coulomb wall:

$$P_{tunnel} \sim e^{-\frac{2\pi Z_1 Z_2}{h v_{rel}}}$$

• Highest reaction probability for light nuclei at high relative velocity!

Solar fusion reactions: The pp-chain





The first step involves the weak interaction, transforming a proton into a neutron, resulting in a very long time scale, i.e. small reaction rates.
This is the reason for the long life time of stars.

- An alternative to this first step involves 3 body collisions, and is therefore very rare: $p + p + e^- \Rightarrow d + v_e$
- Fusion reactions also create the heavier nuclei in the stars
 - \Rightarrow stellar Nucleosynthesis
- The neutrinos from this reaction are the only particles to be observed on earth

The CNO-cycle (Bethe-Weizsäcker-cycle)





- Discovered in 1938, independently by Hans Bethe (Cornell University) and Carl-Friedrich von Weizsäcker.
- Catalytic process at temperatures above 1.5 keV, based on ¹²C.
- Not important in the sun, but for all larger (i.e. hotter) stars.
- This process requires the existence of carbon!
- Net reaction: 4 p \Rightarrow ⁴He + 2 e⁺ + 2 v + 3 γ

For a terrestial energy source we need different fusion reactions!





• The weak interaction makes the pp-chain a rather slow reaction.

=> long lifetime of stars.

- The huge mass of the sun makes up for that easily, still resulting in a large power production.
- However, for power production on earth, the weak interaction has to be avoided.
- For the small volume we can afford, we need faster fusion reactions.

Fusion on earth



d + d
$$\Rightarrow$$
 ³He + n + 3.27 MeV (50%)
or t + p + 4.03 MeV (50%)

d + t \Rightarrow ⁴He + n + 17.59 MeV

 $d + {}^{3}He \Rightarrow {}^{4}He + p + 18.35 \text{ MeV}$

- d = 2 H, Deuterium
 - $t = {}^{3}H$, Tritium the heavy hydrogen isotopes.
- Best choice: the DT-reaction



Fusion reactions, the nuclear part



The fusion cross section can be written as





- **Deuterium** exists with a weight fraction of 3.3•10⁻⁵ in water
 - \Rightarrow static range of billions of years.
- **Tritium** is a radioactive isotope and decays with a half life of 12.33 years:

 $T \rightarrow He + e^{-} + v_{e}$

 \Rightarrow no natural tritium available, but production with fusion produced neutrons is possible:

 $n + {}^{6}Li \rightarrow {}^{4}He + T + 4.8 \text{ MeV}$

 $n + {}^{7}Li \rightarrow {}^{4}He + T + n' - 2.5 MeV$

The latter reaction allows self-sufficient tritium breeding.

• Lithium is very abundant and widespread (in the earth's crust and in the ocean water), sufficient for at least 30 0000 years.

Thermonuclear fusion



High relative velocity of the nuclei is necessary \Rightarrow accelerator? No! Coulomb scattering makes the beams diverge \Rightarrow not efficient



Thermalised mixture of deuterium and tritium at temperatures of some 10 keV is needed \Rightarrow plasma.

Energy distribution of particles in a thermal plasma: Maxwell distribution

$$f(v) = \left(\frac{m}{2\pi kT}\right)^{3/2} \exp\left(-\frac{mv^2}{2 kT}\right)$$

where f(v) is the number of particles in the velocity interval [v, v+dv].

Reaction parameter



Reaction rate per unit volume: $R = n_1 \cdot n_2 \cdot \langle \sigma \cdot v \rangle$ when $\langle \sigma \cdot v \rangle$ is the average of $\sigma \cdot v$ over the velocity distribution, and v is the relative velocity

⇒ Transforming the integration into the center-of-mass sytem yields

$$<\sigma \cdot v> = \frac{4}{(2\pi m_r)^{1/2} (kT)^{3/2}} \cdot \int \sigma(E_r) \cdot E_r \cdot exp(-\frac{E_r}{kT})$$

when E_r is the rel. kinetic energy and m_r is the reduced mass, $1/m_r = 1/m_1 + 1/m_2$.





Lawson Criterion



In 1957 Lawson introduced power balances:

Break-even: The fusion power

equals the loss by radiation,

$$P_{\text{bremsstrahlung}} = c_1 \cdot n_e^2 \cdot Z_{\text{eff}} \cdot (kT)^{1/2}$$

(when $c_1 = 5.4 \cdot 10^{-37}$ Wm³keV^{-1/2}, and $Z_{eff} = \Sigma n_i Z_i^2/n$ is the effective plasma charge), and by transport (diffusion, convection, Charge-Exchange): $P_{loss} = 3 \text{ n kT} / \tau_E$

With $n_D = n_T = n/2$, $T_i = T_e = T$ we find a condition for the fusion product $n\tau T$:

n τ T =
$$\frac{12 (kT)^2}{(s - v) - E_{fus} - 4 c_1 Z_{eff} (kT)^{1/2}}$$

Ignition: The neutrons leave the plasma, the α -particles are confined and heat it. Only their energy should enter the balance! $E_{fus} \rightarrow E_{\alpha}$

Ignition Criteria





A more refined analysis also takes into account the α -particles produced in the fusion reactions, as their production is intrinsically coupled to fusion power (3.53•10¹¹ atoms/s/W).

 \Rightarrow Closed curves parametrized by the normalized He-confinement time ρ_{He} = τ^*_{He} / τ_{E}

Fusion Concepts

Requirement for $n\tau T \Rightarrow 2$ concepts:

- 1) High $\tau \Rightarrow$ Magnetic confinement: A thermal plasma is confined by magnetic fields and heated to high temperature.
- 2) High n ⇒ Inertial confinement: A small frozen fuel pellet is heated and compressed by high power beams: Ignition and burn while its "inertia" keeps it together.

Ignition in a small, central spot (low n), propagating outward into area of high n (low T), spark ignition (Nuckolls et al. 1972)

Problems:

- Uniformity of irradiation and compression,
- Rayleigh-Taylor-Instabilities
- Drivers



Magnetic confinement:

topic of this course

IPP

Hohlraum targets, Indirect drive

Uniformity of the target irradiation can be achieved in so-called Hohlraums:

homogeneous

X-ray radiation fusion capsule Laser beams Au-Hohlraum

The laser heats the inside of a high-Z hohlraum, which then emits thermal radiation (X-rays), which is absorbed with high efficiency.





Drivers I (Lasers)



General requirements:

- Pulse energy: 2-10 MJ
- Pulse duration: 10 ns
- Repetition rate: 1-10 Hz
- Energy gain of the pellet burn should be > 1000

LASER:

- 1) Neodym glass laser:
 - at λ = 1.06 µm, absorption is too small. Improvement by frequency conversion to 530 nm (70%) or 350 nm (50%) in potassium dihydrogen phosphate (KDP) crystals .
 - ε_{driver} < 1% (pumping presently by flashlamps, i.e. white light),
 - \Rightarrow Solid State Diode Pumped Lasers (Yb:S-FAP crystals) with efficiencies up to 20% under development (LLNL, : 50J, 10 Hz, 15ns).
 - repetition rate about 1 pulse/2 2008 hrs.
 - achievements: NOVA, Livermore NIF, Livermore
 - 125 kJ, 10 beams 4.2 MJ, 192 beams

2) KrF gas laser:

- λ= 248 nm
- ε_{driver} ~ 1%, potential for development,
- AURORA, Los Alamos: 10 kJ in 500 ns.

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Drivers II (X-Rays from Z-Pinches)

Generally, Z-Pinches are unstable (sausage-instability):

However,

- they generate strong X-Rays during the collapse,
- mult-wire arrays are more stable, generate even more X-Rays!



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Z pinch wire array

FIG. 1. Z-pinch-driven hohlraum ICF concept. Primary hohlraums 1 cm ta with 2.4 cm diam are placed on the ends of a secondary hohlraum 1.6 c tall containing the capsule. The primary hohlraums have annular pow feeds 0.2 cm in width, and are separated from the secondary hohlraum t transport grids. Shine shields of 0.9 cm diam prevent direct pinch illumin tion of the capsule.





National Ignition Facility, NIF Lawrence Livermore National Laboratory





NIF Construction I





NIF Construction II





NIF Construction III





National Ignition Campaign





High-Energy Deuterium-Tritium Experiments Resume

On Aug. 27, 2011, the NIC team began a new round of high-energy experiments on NIF using cryogenically cooled equimolar (50-50) deuterium-tritium (DT) fuel. In the fourth layered DT experiment, all 192 NIF beams delivered 1.41 MJ of ultraviolet light to the target using a modified pulse based on the results of the recent re-emit and shock-timing experiments. Preliminary estimates indicate that the neutron yield was about 2×10^{14} (200 trillion) and the x-ray emission data showed a small, round core, consistent with earlier symmetry tuning results. ... experiments with laser energies of up to 1.6 MJ in the coming weeks.



Magnetic Confinement

Plasma Physics Dinklage previous talk

Charged particles are confined by magnetic fields



Transport perpendicular to B only from collisions. Particles escape only parallell to B, i.e. at the ends. \Rightarrow bend it to a torus.

Gradient drift requires a rotation of the magnetic field lines

 \Rightarrow magnetic surfaces



Stellarators

Stellarators: Kleiber, Hirsch Thursday





- A poloidal field is created by helical, external.
- Invented in the 50's by
 - L. Spitzer jr. At Princeton.
- + Only external currents,
- + well controllable,
- + stationary operation intrinsic
- problem of nested coils,
- trapped particles unconfined
- \Rightarrow need and potential for optimization
- \Rightarrow modular stellarators

Development of Stellarators





Stellarator WENDELSTEIN 7-X





Major radius: 5.5 m av. Minor radius: 0.53 m Magnetic field: 3 T, superconducting EURATOM approval in March 1996, start of the project: summer 1997, start of assembly: spring 2005, start of operation: 2014/5.

WENDELSTEIN 7-X, the engineers version





WENDELSTEIN 7-X assembly first magnet module, August 2008



WENDELSTEIN 7-X assembly First module in cryostat, November 2009

WENDELSTEIN 7-X assembly Port assembly in 1st module, September 2010

ASDEX Upgrade

R = 1.65 m	a = 0.5 m	κ = 1.6
$B_t \leq 3.5 \text{ T}$	$I_p \le 1.4 \text{ MA}$	$P_{H} \le 28 \text{ MW}$

start of operation in 1991

2/1989

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Divertor

- plasma confinement with nested, closed magnetic surfaces, but
- plasma edge has to be defined either
 - physically by a material limiter, or
 - magnetically by additional poloidal fields, defining a last closed flux surface, the separatrix.
- First successful experiments inASDEX:
 - cleaner plasmas
 - steep edge gradients
 - \Rightarrow H-mode with improved confinement
- Meanwhile divertor is a standard for for power and particle exhaust.
- Stellarators have an intrinsic separatrix

ASDEX Upgrade plasma

Plasma interior at some keV, \Rightarrow X-Rays

Outside the separatrix, some eV, \Rightarrow H α

steep gradients at the separatrix

strong radiation in the divertor

Quantitative criteria

IPP

Power balance of a fusion plasma:

Alpha-particle heating balance losses

 \Rightarrow criteria for

- T \approx 100 Mio K = 10 keV

Quantitative criteria

Power balance of a fusion plasma:

Alpha-particle heating balance losses

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- T \approx 100 Mio K = 10 keV \checkmark
- $n \approx 10^{20} \text{ m}^{-3}$
- $\tau\approx$ 5-10 s

Energy loss in a magnetically confined plasmas

"Classical" picture:

- Energy- and Particletransport on magnetic surfaces free
- Transport perpendicular to magnetic field only through collisions.
- but losses are about 100 x higher than expected!

- \Rightarrow 1. Larger experiments (longer isolation path)
 - 2. "Intelligent experiments" (understand problems and modify)

Empirical scaling of energy confinement

Turbulence-dominated energy loss

Joint European Undertaking

 $\begin{array}{ll} R=2.95\mbox{ m} & a=1.25\mbox{ m} & \kappa & =1.6\\ B_t\leq 3.5\mbox{ T} & I_p\leq 7.0\mbox{ MA} & P_H\leq 30\mbox{ MW}\\ start\mbox{ of operation in 1983} \end{array}$

1997, Mark IIA Divertor

JET DT-Experiments

DT-Experiments only in - JET

- TFTR, Princeton

with world records in JET:

 $P_{fusion} = 16 \text{ MW}$

Q = 0.65

Status of Fusion Research

- Todays tokamak plasmas are close to breakeven,
- The next step (ITER) will ignite or at least operate at high Q (≈10),
- and thereby prove the scientific and technological feasibility of fusion energy.

ITER (latin: the way)	ITER: Lisgo	IPP
<image/>	Friday since Euro and t • Final July 2 R [m a [m] k d I _p [M B [T] T _{nuls}	hational project 1985, started by pe, Japan, Russia, the USA. Design Report in 2001. 1.7 0.35 A] 15.1 5.3 [s] 400
	P _{fusio}	_n [MW] 400

The ITER project

Prototypes of all major components have been built in the R&D program

- to prove the technologies
- to get a reliable costing

segment of the transformer coil

The ITER project

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- ITER site decision taken in June 2005, ITER-contract signed in 2006.
- ITER site: a research site of CEA in Cadarache, near Aix-en-Provence.
- ITER Organization and the Domestic Agencies have been set up.
- The site has been prepared.
- July 2010: Baseline (design, schedule & costs) has been approved.
- New management, simplified structure
- Start of operation is scheduled for 2019

ITER ... finally we get started

ITER ... finally we get started

Schematic fusion reactor

